Experimental and Thermodynamic Studies of Beryllium replacement Materials for CANDU[®] Brazed Joints

K.N. Potter, G.A. Ferrier, and E.C. Corcoran Royal Military College of Canada, Kingston, Ontario, Canada (Kieran.Potter@rmc.ca)

Note: An expanded version of this paper has been submitted to the parallel 35th Annual CNS/CNA Technical Conference.

Abstract

Currently, appendages are joined to CANDU[®] fuel elements via a brazing process, which uses beryllium as the filler material. A potential reduction in the occupational limit on airborne beryllium particulates has motivated research into alternative brazing materials. To this end, the Canadian nuclear industry has funded an initiative to identify and evaluate the suitability of several candidate materials.

This work describes contributions toward the assessment of alternative brazing materials from the Royal Military College of Canada. Thermodynamic modelling was performed to predict the aqueous behaviour of each candidate material in CANDU coolant conditions characteristic of reactor shutdown, and experiments are underway to support modelling predictions. These results will assist in selecting a suitable replacement material for beryllium.

1. Introduction

CANDU fuel sheaths feature two types of appendages: spacers and bearing pads. Spacers are small Zircaloy-4 (Zr-4) pieces, which are attached at the axial centre of each fuel element. The spacers ensure that separation between fuel elements is maintained [1]. Larger Zr-4 appendages called bearing pads are attached to the exterior elements of the fuel bundle. These allow the bundle to slide within the pressure tube and maintain the designed geometry of the bundle within the fuel channel. Specifically, they position the fuel bundles such that the fuel sheaths do not contact the pressure tube, thereby allowing the heat transport system to work effectively. Both appendages are shown in Figure 1.



Figure 1- Bearing pads and spacers attached to fuel sheathing [1]. - 1 of 6-

These appendages are attached to fuel sheaths using a brazing process, with beryllium (Be) as the filler material. This process can create airborne beryllium oxide particulates, which are a health hazard to workers in the manufacturing environment. To address these concerns, the Ontario Ministry of Labour is expected to adopt a 40-fold reduction in the allowable limit on airborne Be particulates [2]. This could interfere with CANDU fuel manufacturing, as current methods may have difficulty achieving compliance with the new regulations. In anticipation of this change, a CANDU Owners' Group (COG) initiative has been launched to investigate the use of alternative filler materials to replace beryllium [3]. Within this initiative, extensive testing of alternative material brazed joints is required. This includes evaluations of joint constructability, mechanical strength, corrosion resistance, high temperature behaviour, and behaviour under irradiation.

This work focuses on evaluating the corrosion resistance of alternative material brazed joints. Thermodynamic modelling was undertaken to generate predictions of corrosion susceptibility for the brazing materials under consideration. Additionally, experiments are underway to supplement modelling predictions and assess the suitability of each candidate material based upon corrosion behaviour in coolant conditions characteristic of reactor shutdown. The results of these studies will assist in determining the most promising candidate brazing material.

2. Modelling of Aqueous Solubility

Thermodynamic models may be used to predict the equilibrium solubility of metals in an aqueous system, at varying conditions of: (i.) pH, (ii.) reduction potential, (iii.) pressure, and (iv.) temperature. Using Gibbs Energy Minimization, the equilibrium phases present in the aqueous system may be calculated, allowing conditions of immunity, passivity, and corrosion susceptibility to be predicted. Models based upon the work of Pourbaix were developed to predict the corrosion behaviour of each alternative material alloy, in order to assist in determining each alloy's suitability for replacing beryllium as a filler material in CANDU brazed joints.

2.1 Theory

Corrosion occurs by electrochemical processes, which are described by the generic reaction (Eq. 1), where *R* is a solid reactant, *P* is an ionic product in solution, H^+ is the hydrogen ion in solution, and e^- are free electrons. The symbols *r*, *w*, *x*, *h*, and *n* represent the reaction coefficients for the corresponding reactants and products.

$$rR + wH_2O \to xP + hH^+ + ne^- \tag{1}$$

The equilibrium reduction potential is given as a function of the activities of participating species by the Nernst equation (Eq. 2):

$$E = \frac{\Delta G}{nF} + \frac{0.0591}{n} \log \frac{(a_P)^x (a_{H+})^h}{(a_R)^r (a_{H_2O})^w}$$
(2)

where E is the reduction potential, ΔG is Gibbs energy change of the reaction (Eq. 1), F is Faraday's constant, a is the activity of species involved, and n is the number of free electrons. It is noted that

ionic activity is a measure of the "effective concentration" of an ion. Given that the activity of H_2O is necessarily equal to 1 (*e.g.*, $a_{H_2O} = 1$), reactant *R* is solid (*e.g.*, $a_R = 1$), and $pH = -\log(a_{H^+})$, the Nernst equation may be rewritten as:

$$E = \frac{\Delta G}{nF} + \frac{0.0591x}{n} \log(a_P) - \frac{0.0591h}{n} \text{pH}$$
(3)

This equation allows the equilibrium activity of ions in solution, a_p , to be calculated as a function of system pH and reduction potential, provided the Gibbs energy change (ΔG) of the dissolution reaction is known. The Gibbs energy change of the reaction is calculated as the sum of the Gibbs energy of formation (G_T) of the products minus that of the reactants, which are determined at the specified temperature using Equation 4:

$$G_T = H_{298K}^o + \int_{298}^T C_P dT - T \left(S_{298K}^o + \int_{298}^T \frac{C_P}{T} dT \right)$$
(4)

Therefore, if standard thermodynamic data for the standard enthalpy of formation (H_{298K}^o), entropy of formation (S_{298K}^o), and constant pressure heat capacity (C_P) are known, Equation 3 may be used to calculate the equilibrium activity of each possible species. At low ion concentrations ($<1 \times 10^{-4}$ mol L⁻¹), activity and concentration are approximately the same, and calculated activity is considered to be equivalent to the solubility limit for that ion [6].

2.2 Modelling Approach

To enable predictions of braze alloy solubility, a model containing thermodynamic data for possible braze alloy solid and aqueous species was established. A literature review was conducted to determine the enthalpy of formation, entropy of formation, and constant pressure heat capacity of the relevant solid and aqueous compounds of Zr, Be, Ni, Fe, and Cr, which comprise the significant constituents of the brazing alloys under consideration. The thermodynamic model was then validated by comparing Pourbaix diagrams produced to those published in the literature [7,8].

2.3 Preliminary Results

As an initial step in screening potential replacement brazing alloys, an assessment of corrosion resistance in conditions simulating a reactor shutdown is required. In these conditions, the coolant temperature is \sim 50°C and the pH is near neutral (~ 6.6).

The aqueous speciation and equilibrium solubility of each significant brazing alloy element was calculated for these coolant conditions, using the developed model. Calculations were performed using FactSage thermochemical modelling software [9]. Results are shown below in Table 1.

Braze Alloy Element	Predominant Aqueous Species	Calculated Solubility Limit $(mol L^{-1})$
Zr	HZrO ₃ [-]	4.90×10^{-12}
Be	Be[2+]	1.31×10^{-12}
Fe	Fe(OH) ₃	4.26×10^{-12}
Cr	Cr(OH) ₃	3.29×10^{-9}
Ni	Ni[2+]	4.00×10^{-4}

Table 1- Calcula	ted solubility of releva	ant brazing alloy eleme	ents at 50°C and pH 6.6

Noting that the boundary between solid and ionic species in Pourbaix diagrams is often taken to be 1×10^{-6} mol L⁻¹, the aqueous solubility limit of the investigated brazing alloy elements was low (less than 1×10^{-8} mol L⁻¹), in all cases except for nickel, where a solubility limit of 4.00×10^{-4} mol L⁻¹ was calculated [7]. This indicates that in these conditions, dissolution of Zr, Be, Fe, and Cr is not thermodynamically favourable, but may be possible for Ni. However, it is noted that this preliminary model does not yet include multi-element compounds such as NiFe₂O₄, which are likely to be relevant in mitigating corrosion in several candidate brazing alloys [10]. Work is currently ongoing to incorporate relevant multi-element compounds into the model.

Additionally, it is noted that the calculated solubility limits are equilibrium predictions, which do not necessarily represent the actual aqueous species concentrations in braze alloy systems. Depending on kinetics, a real system may not reach these predicted dissolved species concentrations. Therefore, experiments will be completed to compare real aqueous species concentrations to those predicted by the thermodynamic model.

3. Corrosion Experiments

In addition to modelling activities, experiments will be performed to investigate the corrosion resistance of alternative brazing materials at conditions characteristic of reactor shutdown. These are intended to support modelling predictions and to supplement the operational conditions testing performed by our project collaborators. An experiment apparatus has been designed, and experiments are currently underway.

3.1 Experiment Set-up

The *shutdown conditions* corrosion experiment is intended to simulate an extended reactor outage. Samples of bearing pad joints constructed with candidate materials will be exposed to water at 50°C, near neutral pH, and dissolved oxygen concentrations up to 3 ppm, for a period of 50 days. The bearing pad samples will then be removed, and properties including weight change, oxide thickness (on the braze alloy, bearing pad, and fuel sheath), hydrogen uptake (of the sheath and bearing pad), braze alloy degradation, and mechanical strength will be evaluated. These data will be used to assess potential corrosion effects on the integrities of the brazed joints and surrounding sheath areas.

An apparatus has been designed to accomplish this experiment (Figure 2 and Figure 3). The brazed joint specimens will be held in a Teflon sample holder, to keep samples electrically isolated from each other. The sample holder will sit in a sealed Pyrex vessel, containing an inert FEP (fluorinated ethylene

propylene) liner to prevent test liquid contamination. Each sample type will be held in a vessel containing the test solution, and the vessels will be held in a temperature-controlled water bath for the duration of the experiment.



Figure 2- CAD model of the corrosion test apparatus.

Figure 3- Exploded view of the test apparatus, showing the glass vessel, Teflon sample holder, and brazed bearing pad specimens.

After the experiment, inductively coupled plasma mass spectrometry (ICP-MS) analysis will be performed to measure the concentration of species that may have leeched from the braze alloy into the test liquid. These results will be compared to aqueous species predictions from thermodynamic modelling activities.

4. Conclusions

Corrosion studies of candidate replacement brazing materials are underway. A preliminary thermodynamic model has indicated that the Ni present in some candidate brazing materials may dissolve in CANDU coolant at shutdown conditions, while other significant braze alloy elements have very low solubility limits and are unlikely to corrode ($< 1 \times 10^{-8}$ mol L⁻¹). Work is ongoing to incorporate multi-element compounds into the model, which are likely to lower the modelled solubility of Ni at these conditions. Additionally, an experimental apparatus has been designed, and experiments are underway to support modelling predictions and assess the corrosion resistance of candidate braze joints in simulated reactor shutdown conditions.

5. Acknowledgements

The authors acknowledge the financial support of Natural Science and Engineering Research Council of Canada, the CANDU Owners' Group, and University Network of Excellence in Nuclear Engineering for financial support of this research effort. The authors are appreciative of the technical expertise and guidance from representatives of the Canadian Nuclear Laboratories (formerly AECL), Cameco Fuel Manufacturing, GE-Hitachi, Bruce Power, and Ontario Power Generation. These include F.C. Dimayuga, R. Harman, R. Scrannage, S. Palleck, T. Onderwater, E. Lu, and T. Daniels.

6. References

- [1] A. Manzer, "Introduction to CANDU Fuel", Presented to USNRC, Washington, DC. 4 September 2003. Available at http://pbadupws.nrc.gov/docs/ML0325/ML032530144.pdf
- [2] "Alert: Workplace Beryllium Exposure". Ontario Ministry of Labour, November 2010. Available at https://www.labour.gov.on.ca/english/hs/pubs/alerts/a21.php
- E.C. Corcoran, T. Daniels, J. Harmsen, E. Lu, S. Palleck, A. Pant, T. Onderwater, and F. Dimayuga, "Status of The Beryllium Replacement Project", Presentation at the 12th International Conference on CANDU Fuel, Kingston, Ontario, Canada, 15-18 September 2013.
- [4] E.D. Verink, "Simplified procedure for constructing Pourbaix diagrams", *Uhligs Corrosion Handbook*, Ch. 7, pp. 93-102, 2011.
- [5] M. Moran, H. Shapiro, D. Boettner, and M. Bailey, "Fundamentals of Engineering Thermodynamics", John Wiley and Sons, 1 Dec, 2010.
- [6] S. Lower, "All About Electrochemistry: The Nernst Equation", Simon Fraser University 2012. Available at http://www.chem1.com/acad/webtext/elchem/ec4.html
- [7] M.H. Kaye and W.T. Thompson, "Computation of Pourbaix Diagrams at Elevated Temperatures", *Uhligs Corrosion Handbook*, Ch. 9, pp. 111-122, 2011.
- [8] B. Beverskog and I. Puigdomenech, "Revised Pourbaix Diagrams for Nickel at 25-300C", *Corrosion Science, Vol. 39, 1997.*
- [9] C. W. Bale, E. Bélisle, P. Chartrand, S. A. Decterov, G. Eriksson, K. Hack, I. H. Jung, Y. B. Kang, J. Melançon, A. D. Pelton, C. Robelin and S. Petersen, *FactSage Thermochemical Software and Databases Recent Developments*, Calphad, vol. 33, pp 295-311, 2009 <www.factsage.com>
- [10] W.T. Thompson, M.H. Kaye, C.W. Bale, and A.D. Pelton, "Pourbaix Diagram Construction for Multi-element Systems", *Uhligs Corrosion Handbook*, Ch. 8, pp. 103-110, 2011.